





Concept of the UCN Source at the WWR-K Reactor (AISUN)

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ISINN-31, Dongguan, 26-30 May, 2025



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Presentation Plan

1. Brief history of helium UCN converters

- Initial UCN source concept based on helium
- Other UCN source concept based on helium

2. Why WWR-K as a unique UCN source?

- *WWR type reactors*
- WWR-K reactor

3. Concept of the UCN source at the WWR-K reactor

- *Basic concept of UCN source*
- Estimates of the 8.9 Å neutron fluxes
- *Estimates of UCN production*
- Extraction and transport of UCNs

4. Investigations on improvement of UCN concept at WWR-K reactor

- Investigation of the cooling system with a heat conducting wall
- Investigation of wall coatings with high critical energy
- Investigation of focusing neutron guides of UCN

5. Conclusion





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Vertical cross section TRIGA Mark II reactor in Vienna

Golub, R. Ultracold neutrons (UCN) at a Triga reactor. Nucl. Instrum. Methods Phys. Res. 1984, 226, 558–559.

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Schreyer, W.; Davis, C.A.; Kawasaki, S.; et al. Nucl. Instrum. Methods A 2020, 959, 163525.



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| Accelerator | 19.3 kW |
|----------------------------------|------------------------------------|
| power | |
| P _{UCN} | (1.4-1.6)·10 ⁷ UCN/s |
| ρ_{UCN} in exp. set. | 220 UCN/cm ³ |





Other helium UCN source concepts

✓ WWR-M PNPI UCN source concept



A P Serebrov et al 2017 J. Phys.: Conf. Ser. 798 012147

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Reactor power 16 MW P_{UCN} (6-8)·10⁷ UCN/s ρ_{UCN} in exp. set. 10⁴ UCN/cm³

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WWR type reactors





WWR-SM (Uzbekistan, 1959-now) **10 MW**

WWR-M (Gatchina, RF, 1959-2015) WWR-M (Ukraine, 1960-2015) **15 MW**



WWR-Ts (Obninsk, RF, 1964) **15 MW**



2016



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10 MW

WWR-K (Kazakhstan, 1967-now), Conversion, **6 MW**



First UCN

Lushchikov V. I., Yu.N.Pokotilovskii, A.V.Strelkov, F.L.Shapiro. JETP Lett. 9 23 (1969)





F. L. Shapiro V. I. Lushikov

1968 **IBR 6 кW** OBSERVATION OF ULTRACOLD NEUTRONS

Joint Institute for Nuclear Research Submitted 18 November 1968 ZhETF Pis. Red. 9, No. 1, 40 - 45 (5 January 1969)



'ig. 1. Diagram of setup. 1- IBR reactor; 2, 3 - moderator (2 - paraffin, 3 - polyethylene ayer 1 mm thick); 4 - copper tube, 9.4 cm i.d., total length 10.5 m; 5 - aluminum tube; 6 opper-foil cylinder; 7 - shield (paraffin with boron carbide); 8 - 2-m concrete wall of rector chamber; 9 - detector shield (paraffin); 10 - tube filling and evacuating system; 11, .2 - detectors (FEU-13 with layers of ZnS or ZnS + Li compound); 13 - copper shutter (gap etween shutter and detector < 1 mm); 14 - shutter mechanism; 15 - trap for direct beam.



A. V. Strelkov

Ye. N. Pokotilovsky

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V. I. Lushchikov, Yu. N. Pokotilovskii, A. V. Strelkov, and F. L. Shapiro



First UCN in Kazakhstan

Akhmetov, E.Z.; Kaipov, D.K.; Konks, V.A.; Lushchikov, V.I.; Pokotilovskii, Y.N.; Strelkov, A.V.; 474 Shapiro, F.L. Generation of ultracold neutrons at the stationary WWR-K reactor. At. Energy 1974, 475 37(1), 35–38. (In Russ.)



Kaipov D.K.

Akhmetov E.Z.



Shapiro F.L



Strelkov, A.V.



Lushchikov V.I



Pokotilovskii, Y.N

Получение ультрахолодных нейтронов на стационарном реакторе ВВР-К

АХМЕТОВ Е. З., КАИПОВ Д. К., КОНКС В. А., ЛУЩИКОВ В. И., ПОКОТИЛОВСКИЙ Ю. Н., СТРЕЛКОВ А. В.,

1973



Р н с. 1. Схема экспериментальной установки:

наполнения и контроля давления гелия.



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«АТОМНАЯ ЭНЕРГИЯ», Т. 37, ВЫП. 1, ИЮЛЬ



Reactor WWR-K



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Flux density $2.2 \cdot 10^{14} \text{ n/cm}^2 \cdot \text{s}$



Basic concept of UCN source



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Project AISUN Almaty Source of Ultracold Neutron

Article

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Concept of the UCN Source at the WWR-K Reactor (AISUN)

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- Department of Nuclear Engineering, North Carolina State University, Raleigh, NC 27695, USA
- Frank Laboratory of Neutron Physics, Joint Institute for Nuclear Research, 6 Joliot Curie, Ru-141980 Dubna, Russia
- Institut Max von Laue-Paul Langevin, 71 Av. des Martyrs, F-38042 Grenoble, France 4
- Department of Physics, North Carolina State University, Raleigh, NC 27695, USA

Abstract: We present the concept of the ultracold neutron (UCN) source with a superfluid helium-4 converter located in the thermal column of the WWR-K research reactor in Almaty, Kazakhstan. The conceptual design is based on the idea of an efficient system for accumulating UCNs in the source and transporting them to experimental setups. We propose to achieve an increase in the UCN density in the source by separating the heat flow and the UCN flow from the source, lowering the temperature of the SF 4He converter below ~ 1 K and the coefficient of UCN loss in the walls of the accumulation trap. The temperature decrease is expected to be achieved by increasing the efficiency of the helium cooling system, as well as by reducing the thermal load on the UCN accumulation trap walls. A large total number of accumulated UCNs is expected to be achieved due to the high critical velocity of the wall material of the accumulation trap. The implementation of such a design critically depends on the availability of materials with specific UCN and cryogenic properties. This paper describes the conceptual design of the source, discusses its implementation methods and material requirements, and plans for material testing studies.

Special issue in Materials, MDPI, devoted to Workshop in Kazakhstan (2024):

New Materials for Neutron Sources and Instruments

https://www.mdpi.com/journal/materials/special issues/1XF70W3M3M https://indico.inp.kz/event/3/



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Estimates of the 8.9 Å neutron fluxes

Goorley, J.T.; et al. Initial MCNP6 Release Overview - MCNP6 version 1.0. LA-UR-13-229342013.

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Differential neutron flux density averaged over the volume of SF ⁴He trap.

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 $dJ/d\lambda$ (8.9 A) = 1.62 · 10¹⁰ cm⁻² s⁻¹ Å⁻¹



Estimates of UCN production

| ^{Be} R= 4.55·10 ⁻⁸ · dJ/d λ (8.9 A) cm ⁻³ s ⁻¹ | $\rho_{UCN} = \tau_{stor}^0 \cdot \mathbf{R} = \tau_{stor}^0 \cdot \mathbf{P}_{UCN} / \mathbf{V}$ | | |
|--|---|---------------|----------------------|
| Let UCNs are produced in a trap with Be-coated walls | 1 1 1 1 | | |
| Spectrum will be as follows | $\frac{\tau_{stor}^{0}}{\tau_{He}} - \frac{\tau_{wall}}{\tau_{wall}} + \frac{\tau_{\beta}}{\tau_{\beta}}$ | | |
| $dR(E)/dE \sim \sqrt{E}$ | He temperature, K | $	au_{He},$ s | $	au_{stor}^{0}$, s |
| The rate of production of UCNs | 1.2 | 35 | 27 |
| | 1.1 | 70 | 44 |
| $R(E_{lim}) \sim (E_{lim} - E_{He})^{3/2}$ | 1.0 | 130 | 62 |
| | 0.9 | 260 | 82 |
| The production rate of the UCN source | 0.8 | 610 | 100 |
| $P_{UCN} = RV \rightarrow {}^{Be}P_{UCN} = 2.6 \cdot 10^7 UCN/s$ | | | |



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$$= au_{stor}^0 \cdot \mathbf{P}_{\mathrm{UCN}} / \mathbf{V}$$

$$-rac{1}{ au_{wall}}+rac{1}{ au_{eta}}$$

ρ_{UCN} (0.9 K) = 6.1 · 10⁴ UCN/ cm³



Estimation of the density of UCNs in the experimental setup

The experimental setup has the same volume as the source and has the same storage time τ_{stor}^0 as it is in the source

$$\rho_{exp} = \rho_{max}/8$$

Lost factor in the source: 2 Lost factor in the neutron guide: 2 Lost factor in the experimental setup with same volume and no SF ⁴He: 1.65

$$\rho_{exp} = \rho_{max}/6.6 = 9.2 \cdot 10^3 \, \text{UCN/cm}^3$$

Corrections

Factor 1: cutoff of the UCN spectrum on the side of high energies, $k_1 = 0.81$ Factor 2: decrease in the exit flux from the source, $k_2 = 0.88$ Factor 3: 0.1 mm thick separating Al foil, $k_3 = 0.74$

$$\rho_{exp} = k_1 k_2 k_3 \rho_{exp}^0 = 4.9 \cdot 10^3 \text{ UCN/cm}^3$$







Parameters of different concepts

| Improved concept of the UCN source | Basic concept of |
|--|---------------------------|
| Flux density (8.9 Å): $dJ/d\lambda = 1.62 \cdot 10^{10} \ cm^{-2} \ s^{-1} \ \text{\AA}^{-1}$ | Flux density (dJ/dλ |
| SF ⁴ He vessel wall material: AlBeMet | SF ⁴ He vessel |
| SF ⁴ He vessel volume: V=35 J | SF ⁴ He vessel |
| T = 0.9 K | SF ⁴ He temper |
| UCN density in the source: $\rho \sim 6.1 \cdot 10^4 \ cm^{-3}$ | UCN density i |
| UCN density in the exp. setup : $\rho \sim 5 \cdot 10^3 cm^{-3}$. | UCN density i |



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of UCN source

- (8.9 Å): $\lambda = 1.62 \cdot 10^{10} \ cm^{-2} \ s^{-1} \ \text{\AA}^{-1}$
- el wall material:

Al

l volume:

V=35 l

erature :

T = 1.25 K

- $\rho \sim 2.1 \cdot 10^4 \ cm^{-3}$
- in the exp. setup: $\rho \sim 1.7 \cdot 10^3 \ cm^{-3}$.





Investigations on improvement of UCN concept at WWR-K reactor

- > Investigation of the cooling system with a heat conducting wall
- > Investigation of wall coatings with high critical energy

Modeling and Optimization of Experimental Setup Geometry for Measuring Ultracold **Neutron Loss Factors Using Gravitational Spectroscopy**

Investigation of focusing neutron guides of UCN

Calculation for Improving the Efficiency of Ultracold Neutron Transport Using Monte **Carlo Method**







Speaker: Zhanibek Kurmanaliyev

Speaker: Khac Tuyen Pham





Conclusion

- > The basic concept of UCN source based on helium is well developed and studied
- > The WWR-K reactor is ideal place for helium UCN source
- > The improved concept of UCN source based in helium is developed
- Additional investigations should be done to achieve an increase in the UCN density







Estimates of the 8.9 Å neutron fluxes

Thank you for your attention!



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